

The Dicalcium Phosphate production in the NORM context: study of the radiological characterization and dose assessment to workers

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OUTLINE

INTRODUCTION

The phosphate industry and the Dicalcium Phosphate production

The Spanish legal framework concerning NORM

AIMS OF THE STUDY

SAMPLING AND ANALYTICAL METHODS

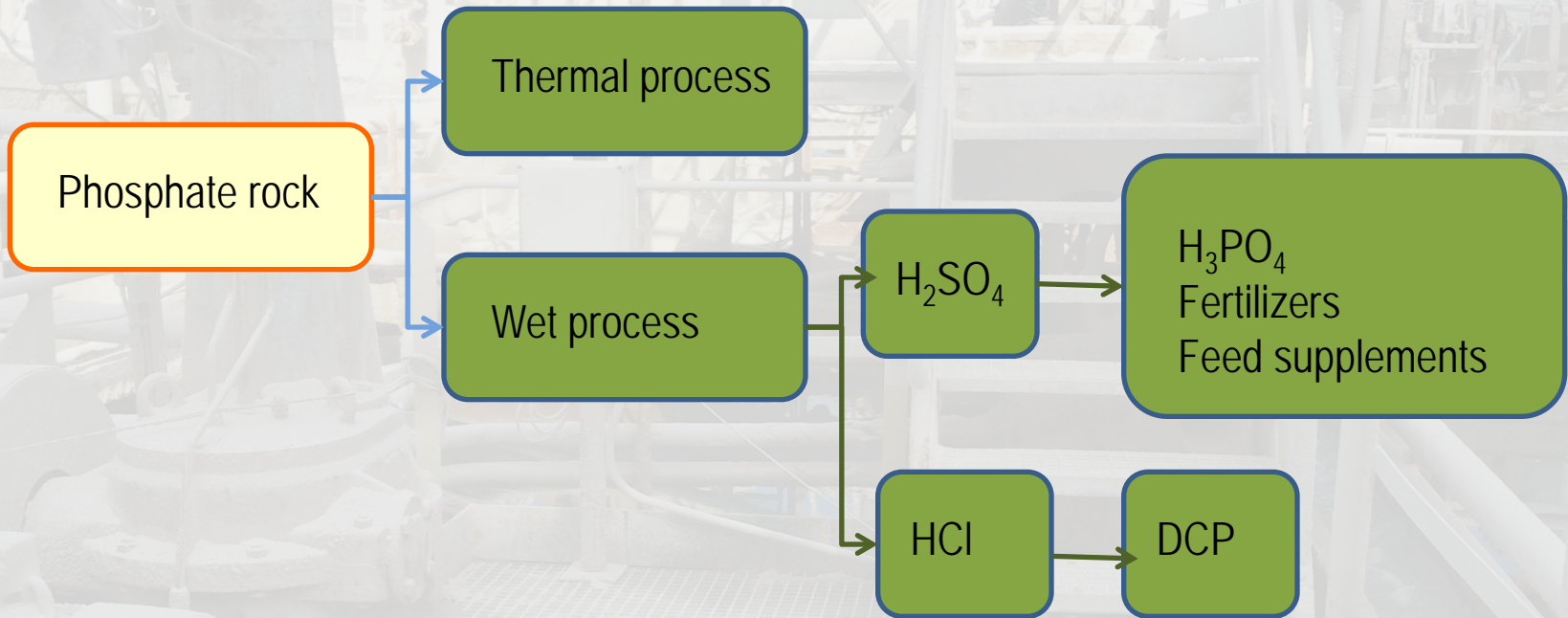
RESULTS on:

RADIOLOGICAL CHARACTERIZATION

DOSE ASSESSMENT TO WORKERS

CONCLUSIONS & FUTURE WORK

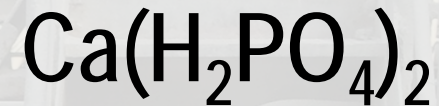
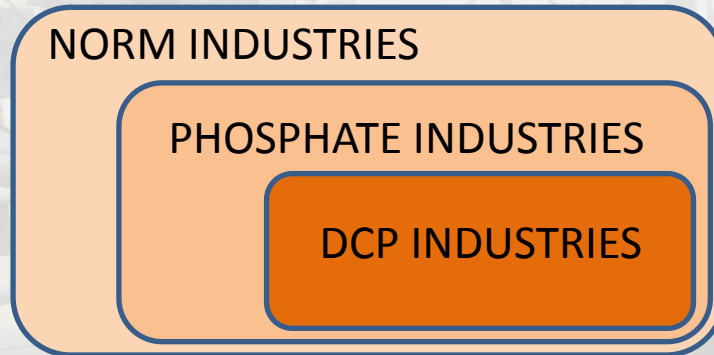
THE PHOSPHATE INDUSTRY



Mineral apatite (francolite or carbonate-fluorapatite). Significant quantities of **naturally occurring radionuclides**; substitution of Ca²⁺ by U⁴⁺ :

- Sedimentary phosphate rocks: **1500 Bq·kg⁻¹** in average
- Igneous deposits: **70 Bq·kg⁻¹**.

THE Dicalcium Phosphate PRODUCTION



- Inorganic feed supplement
- Classified as **feed material** by the European Commission Regulation.
- **Calcium and phosphorus** feed supplement for domestic animals (cattle, poultry, beef, sheep). 18% of P and between 25-30% of Ca.
- High calcium availability (93%).

THE Dicalcium Phosphate INDUSTRY & NORM

NORM industries (IAEA 2006):

- Extraction of REE
- Production and use of thorium and its compounds
- Production of niobium and ferro-niobium
- Mining ores other than uranium ore
- Production of oil and gas
- Titanium dioxide pigments
- **Phosphate industry**
- Zircon and zirconia
- Production of tin, copper, aluminium, zinc, lead, iron and steel
- Combustion of coal
- Water treatment

Thermal phosphorus production
Phosphate fertilizers
Phosphoric acid production
Dicalcium phosphate production

SPANISH LEGAL FRAMEWORK IN NORM

EURATOM 29/96

RD 783/2001

RD 1439/2010

1- The industry holder **MUST** perform the studies to show whether there is an increasing dose to workers and to the public due to the industrial activity.

2- The industry holder **MUST** declare its industrial activity to the Authorities.

3- This RD is also extended to the wastes storage and handling.



Instructions: Nuclear Security Council (CSN)

SPANISH LEGAL FRAMEWORK IN NORM

Instructions: Nuclear Security Council (CSN)

- Effective dose to workers
 - **< 1 mSv·y⁻¹**: no further control is necessary.
 - **1 – 6 mSv·y⁻¹**: low-level control is necessary.
 - **> 6 mSv·y⁻¹**: advanced control is necessary.
- ²²²Rn measures in working areas.
 - **600 Bq·m⁻³** average annual concentration.
 - **600 – 1000 Bq·m⁻³**: low-level control is necessary.
 - **> 1000 Bq·m⁻³**: advanced control is necessary.

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AIMS OF THE STUDY

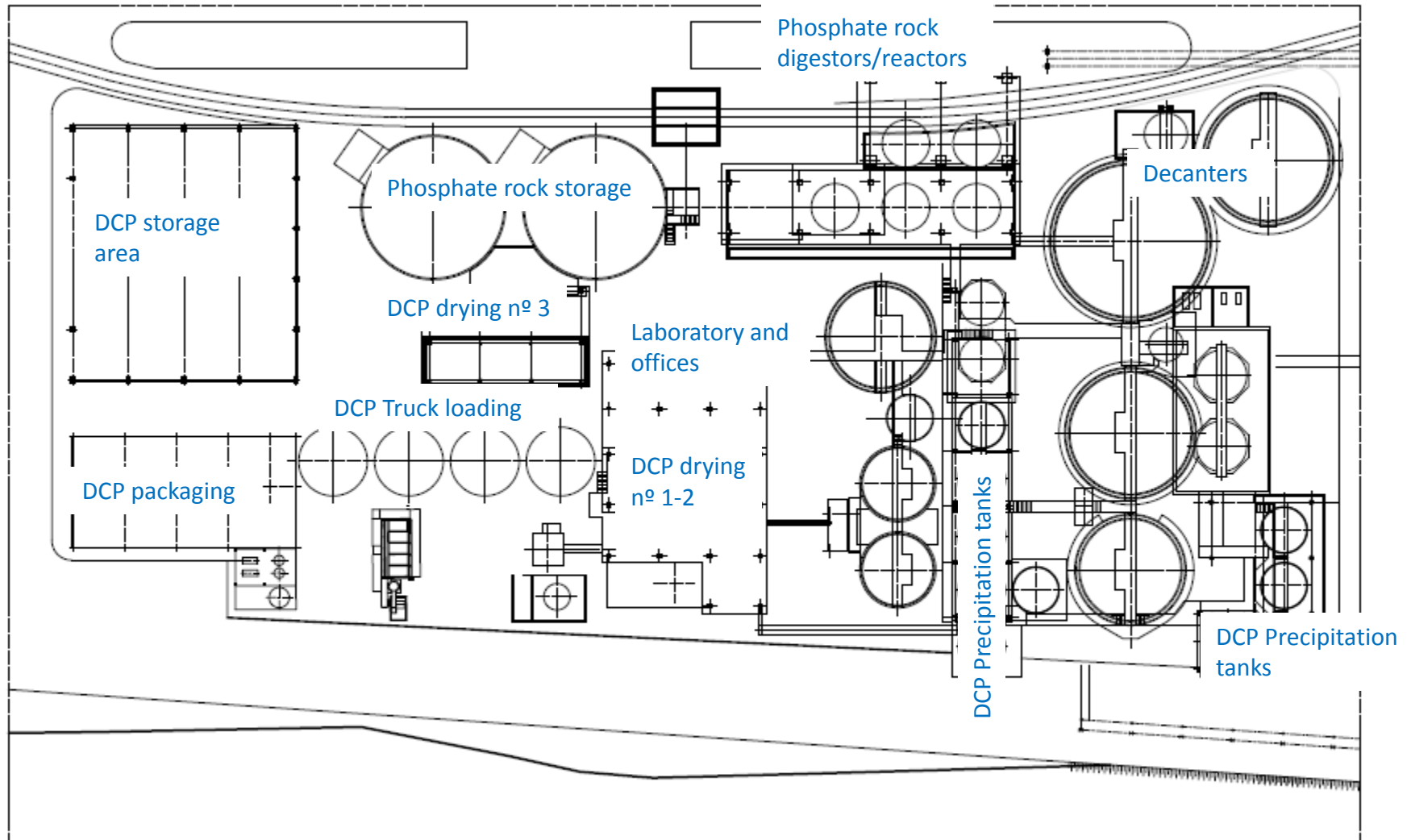
RADIOLOGICAL CHARACTERIZATION:

- Characterize the raw material, products and by-products (^{226}Ra , ^{210}Pb and ^{210}Po).
- Assess the temporal variability.
- Evaluate the radionuclide fluxes (^{226}Ra , ^{210}Pb and ^{210}Po).

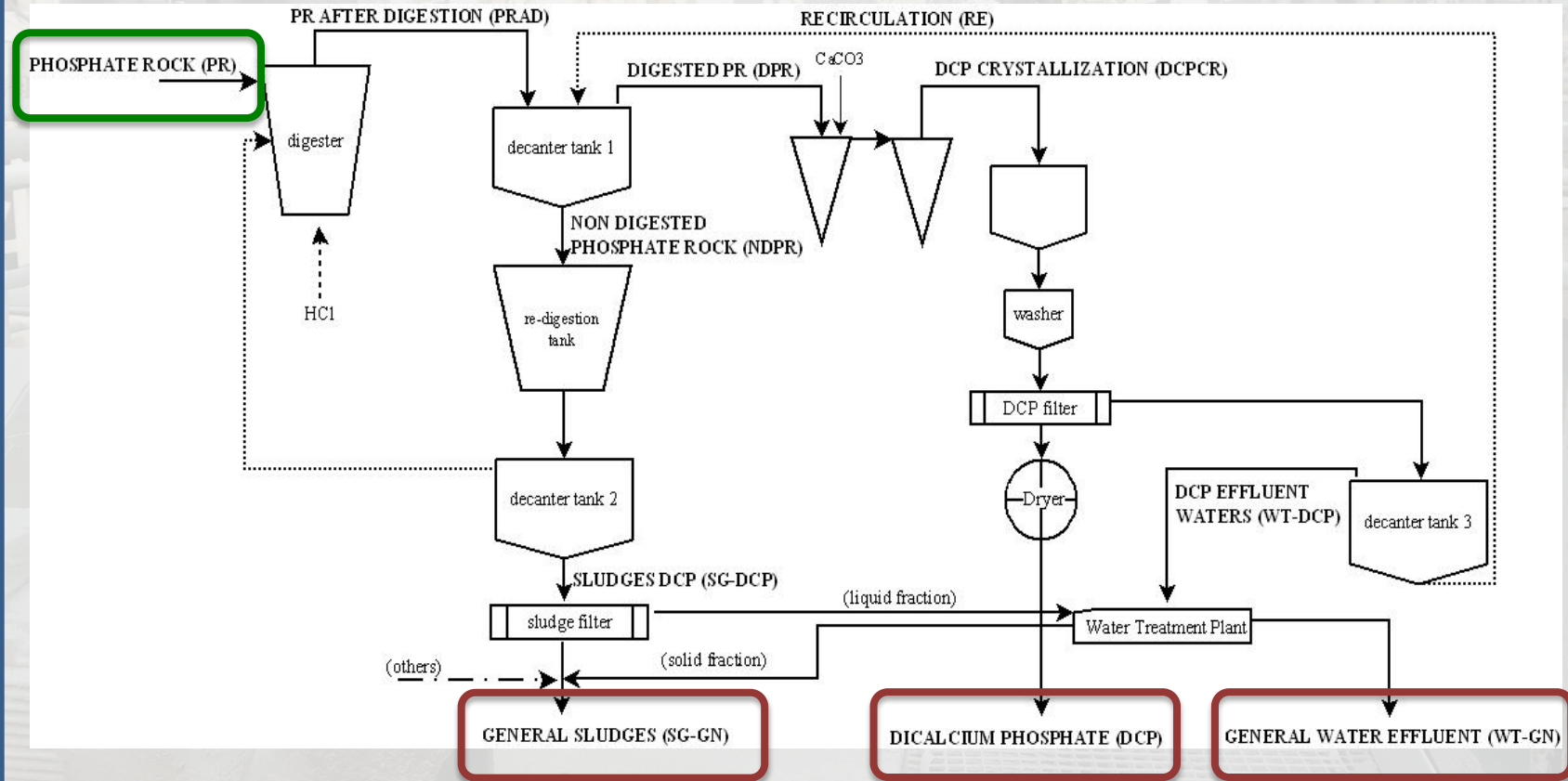
DOSE ASSESSMENT TO WORKERS:

- Study the potential annual dose to workers ($1 \text{ mSv}\cdot\text{y}^{-1}$ ¿?).
- Quantify the doses during the cleaning and maintenance of particular areas in the production process.
- Study the ^{222}Rn concentrations ($600 \text{ Bq}\cdot\text{m}^{-3}$ ¿?).

DCP plant description



RADIONUCLIDE CHARACTERIZATION



RADIOLOGICAL CHARACTERIZATION:

1. Inputs (phosphate rock) and outputs (waters, sludges and DCP).
2. Temporal variability: 1 sample per week during 2 months.

DOSE ASSESSMENT

Gamma dose rate:

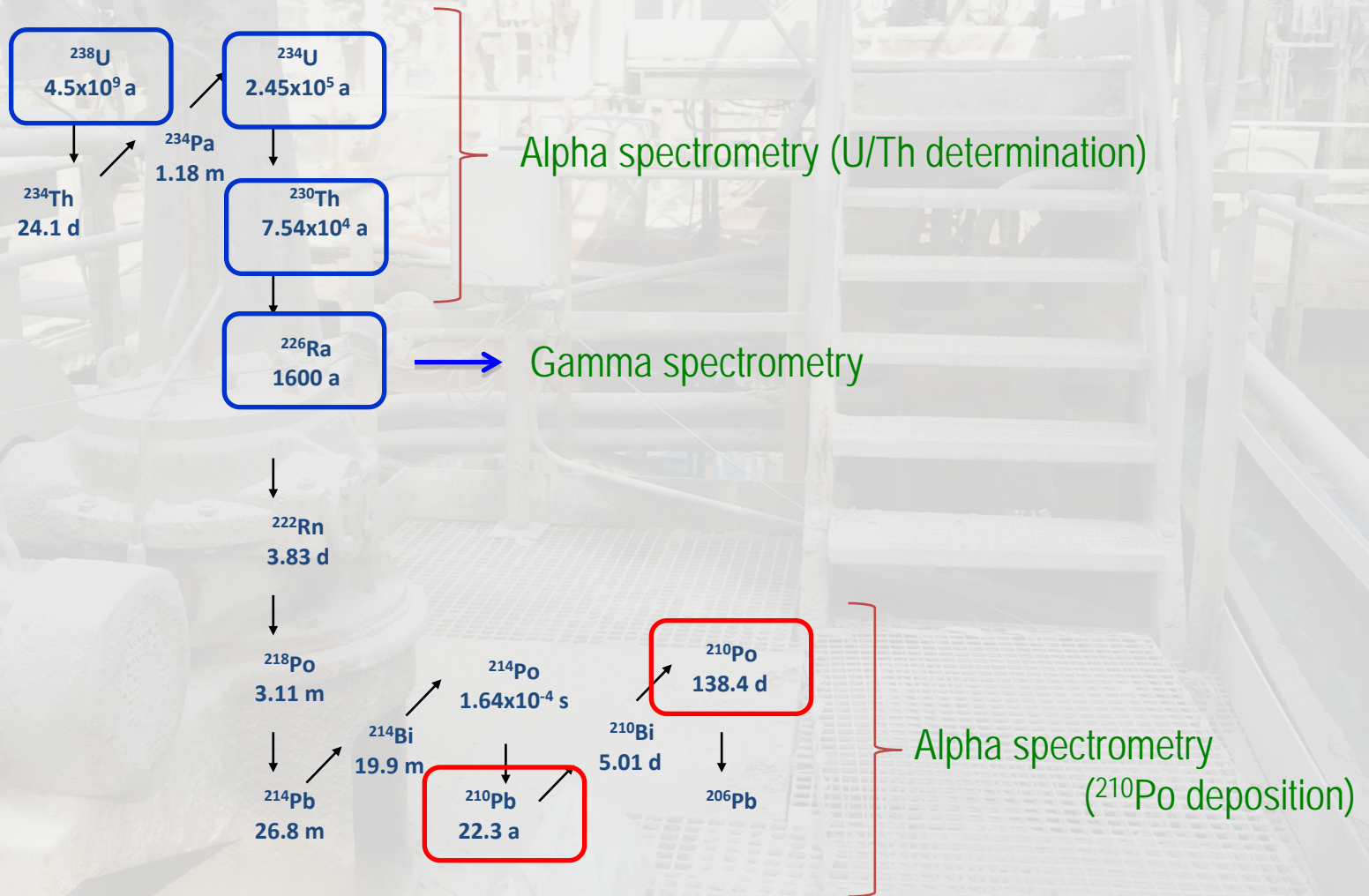
- Area dosimetry (Eberline, Inspector 1000)
- Maintenance and cleaning of the digestors (Personal dosimeter).

Inhalation dose rate:

- Rn measurements (Rad7)
- Dust assessment (RADECO).



ANALYTICAL METHODS



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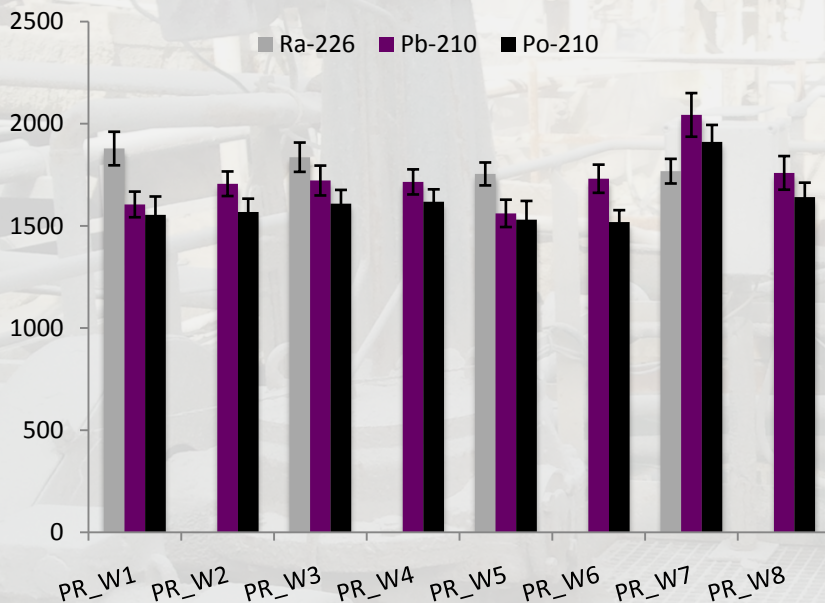
RADIOLOGICAL CHARACTERIZATION

DOSE ASSESSMENT TO WORKERS

CONCLUSIONS AND FUTURE WORK

^{226}Ra , ^{210}Pb & ^{210}Po SPECIFIC CONCENTRATIONS

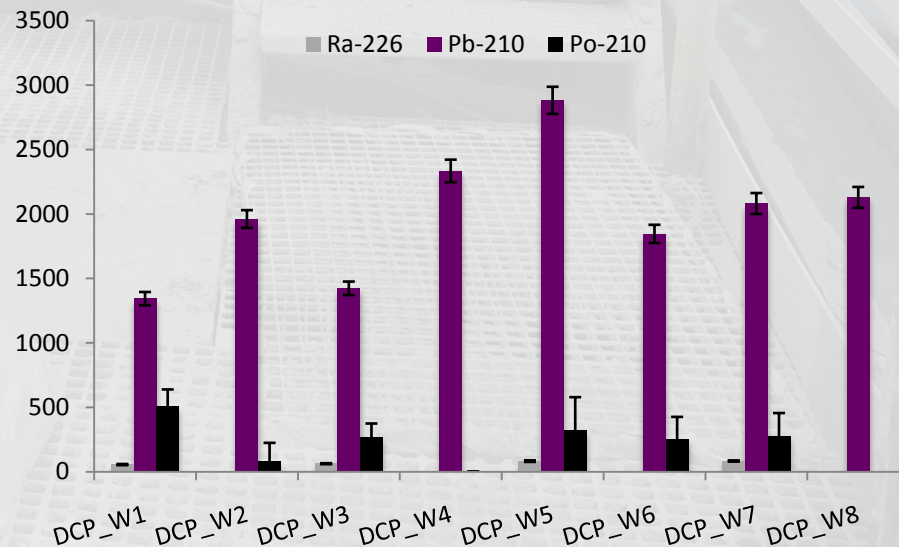
PHOSPHATE ROCK



^{226}Ra : $1809 \pm 59 \text{ Bq} \cdot \text{kg}^{-1}$
 ^{210}Pb : $1731 \pm 143 \text{ Bq} \cdot \text{kg}^{-1}$
 ^{210}Po : $1620 \pm 126 \text{ Bq} \cdot \text{kg}^{-1}$

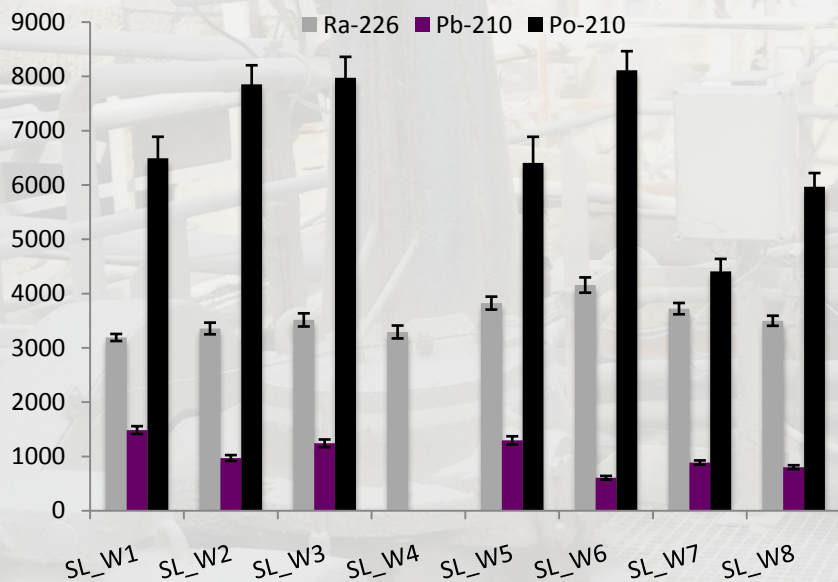
^{226}Ra : 56 – 84 Bq·kg⁻¹
 ^{210}Pb : 1343 - 2882 Bq·kg⁻¹
 ^{210}Po : 79 - 507 Bq·kg⁻¹

DICALCIUM PHOSPHATE



^{226}Ra , ^{210}Pb & ^{210}Po SPECIFIC CONCENTRATIONS

INDUSTRIAL SLUDGES



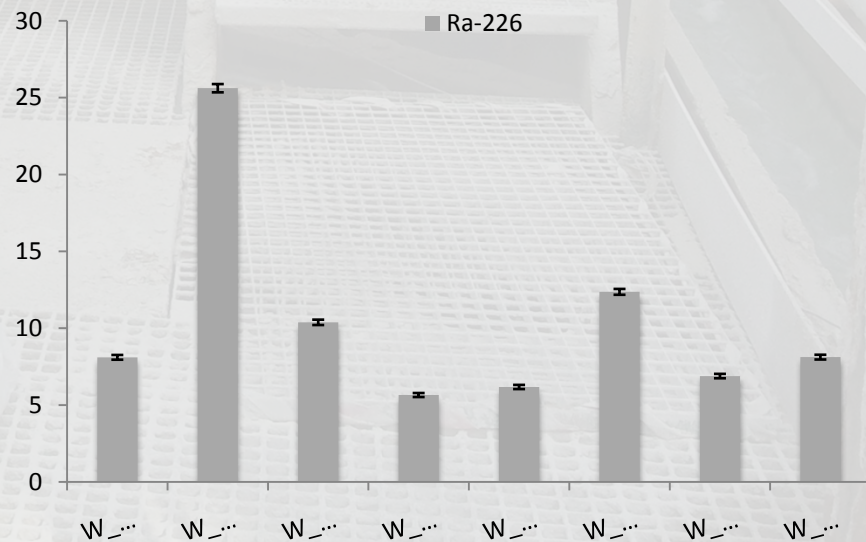
^{226}Ra : 3191 - 4156 $\text{Bq}\cdot\text{kg}^{-1}$

^{210}Pb : 606 - 1485 $\text{Bq}\cdot\text{kg}^{-1}$

^{210}Po : 4407 - 8111 $\text{Bq}\cdot\text{kg}^{-1}$

^{226}Ra : 6 - 25 $\text{Bq}\cdot\text{L}^{-1}$

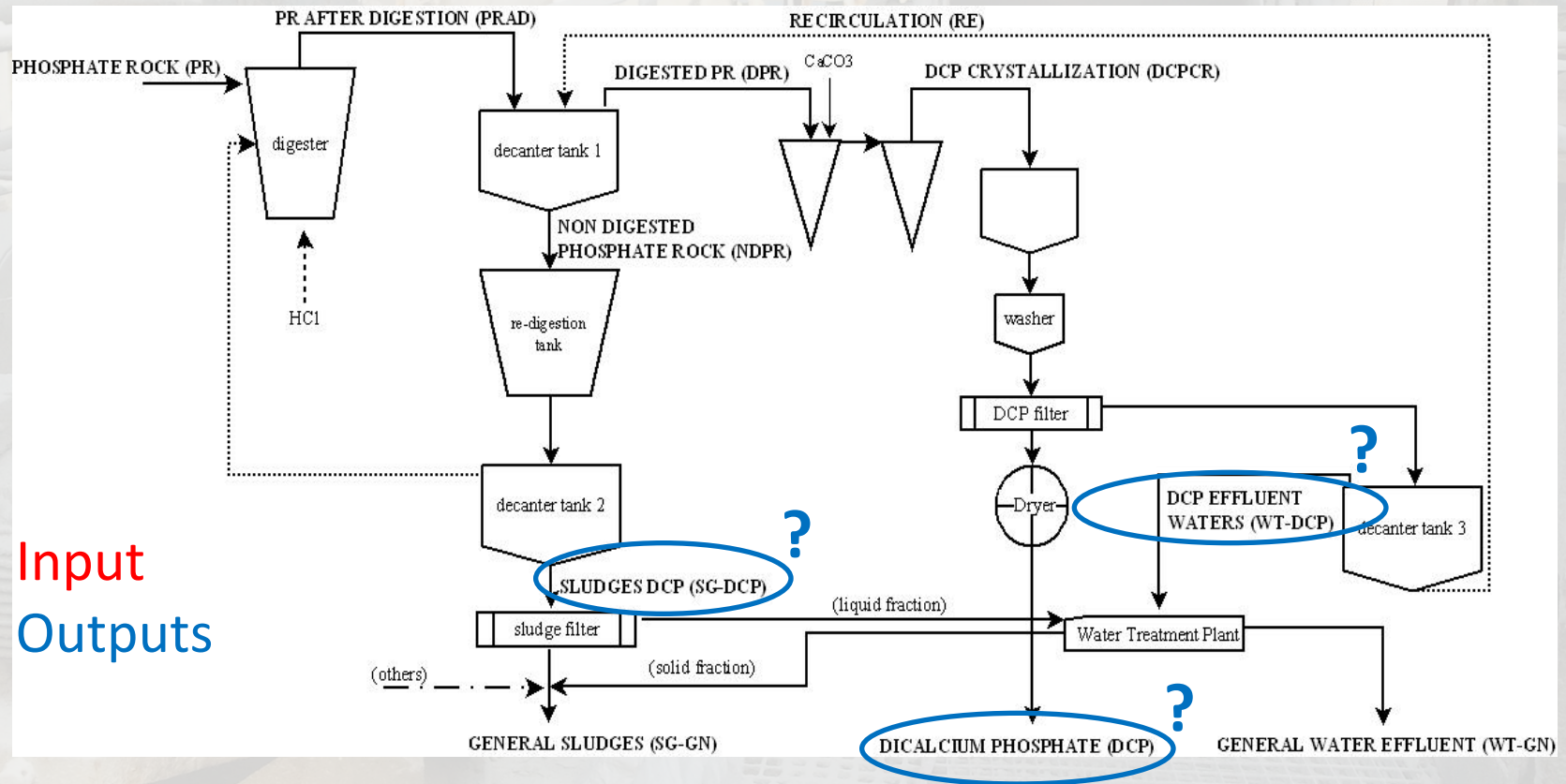
INDUSTRIAL WATERS



FLUXES OF RADIONUCLIDES

100% kBq·h⁻¹

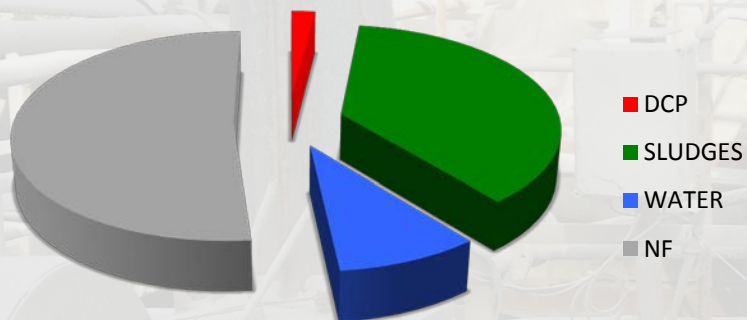
²³⁸U and daughters



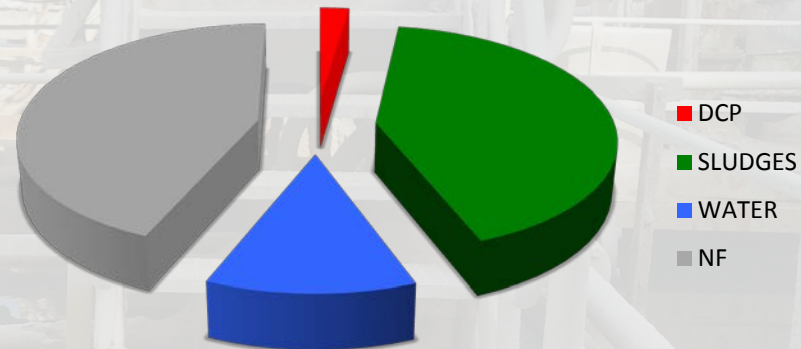
Input
Outputs

OUTPUT FLUXES OF ^{226}Ra

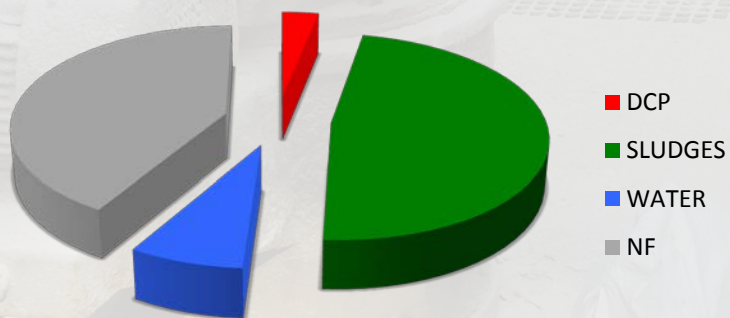
^{226}Ra W1



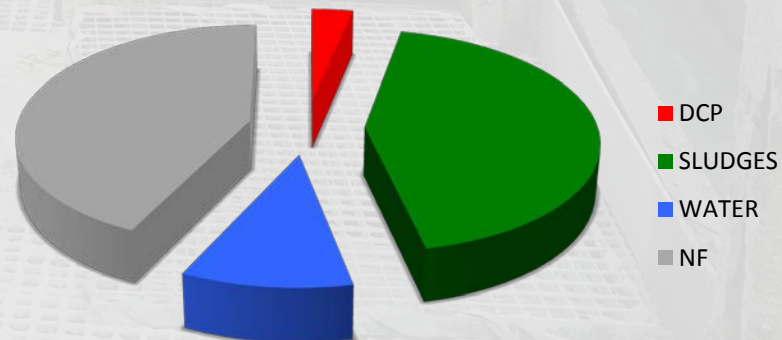
^{226}Ra W3



^{226}Ra W5



^{226}Ra W7

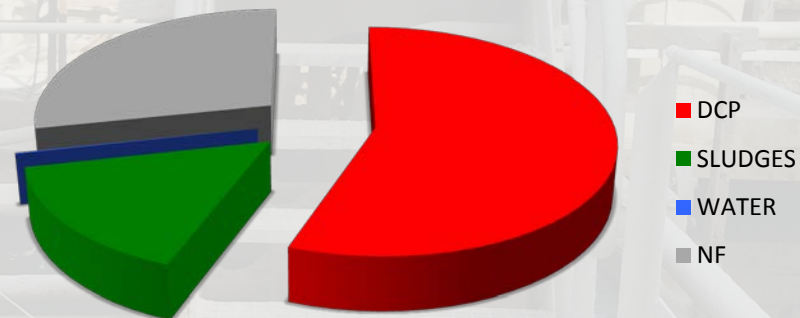


OUTPUT FLUXES OF ^{210}Pb

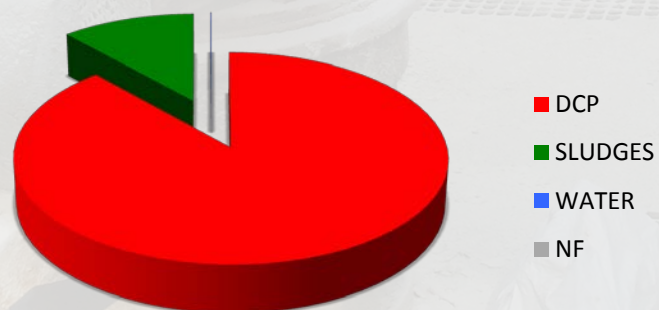
^{210}Pb W1



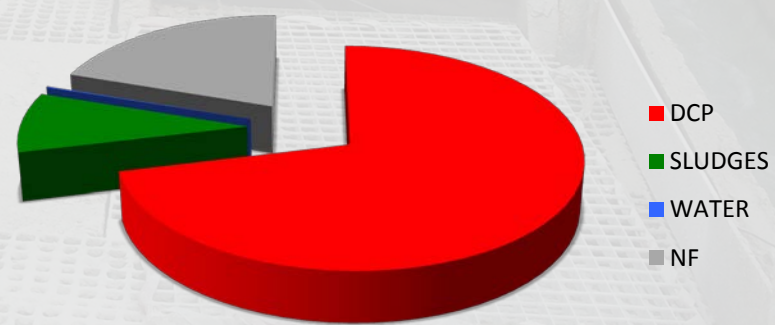
^{210}Pb W3



^{210}Pb W5



^{210}Pb W7

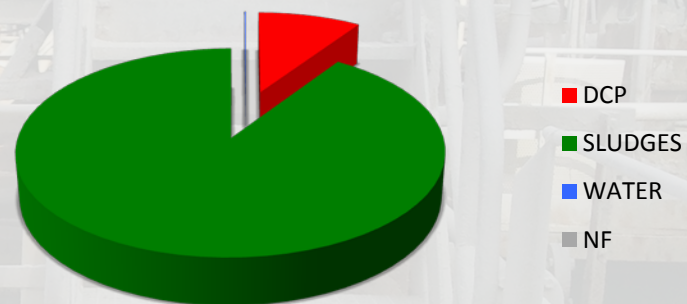


OUTPUT FLUXES OF ^{210}Po

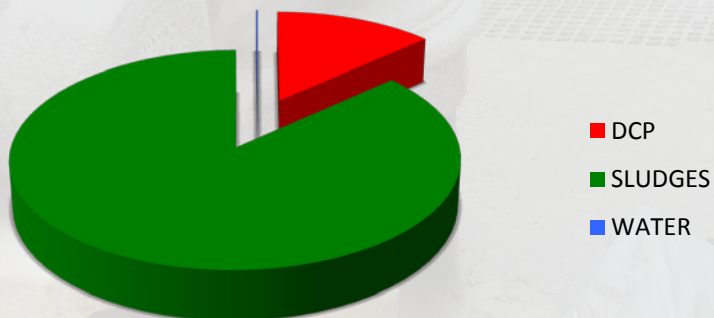
^{210}Po W1



^{210}Po W3



^{210}Po W5



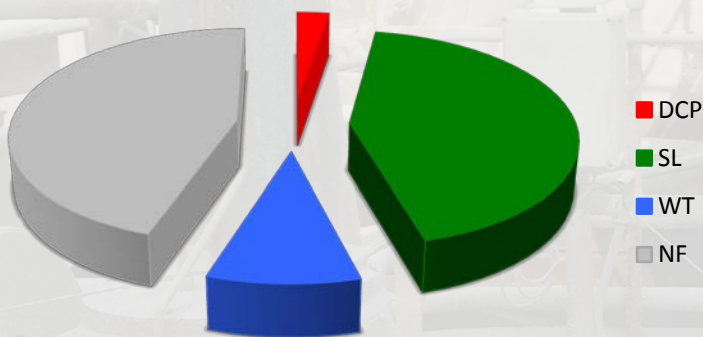
^{210}Po W7



AVERAGE OUTPUT FLUXES (weeks 1, 3, 5 & 7)

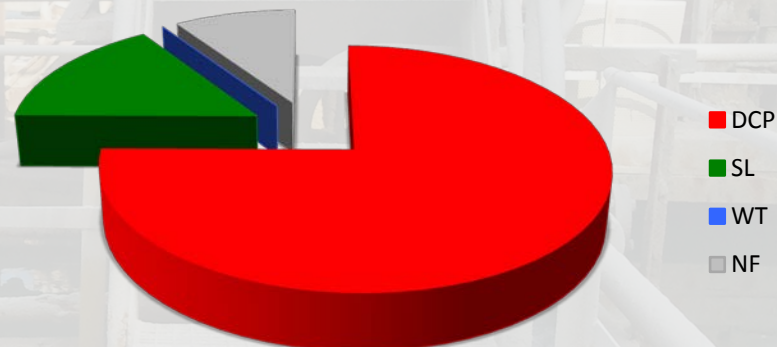
Output to Input Ratio (OIR)

Ra-226



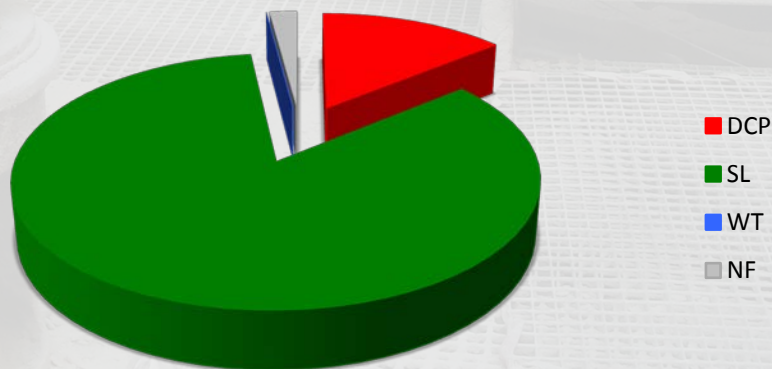
$0,55 \pm 0,04$

Pb-210



$0,91 \pm 0,21$

Po-210



$0,98 \pm 0,25$

RADIOLOGICAL CHARACTERIZATION: things to take into account....

Temporal variability in:

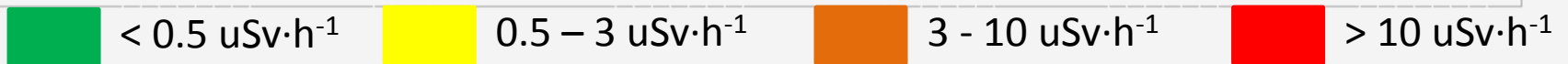
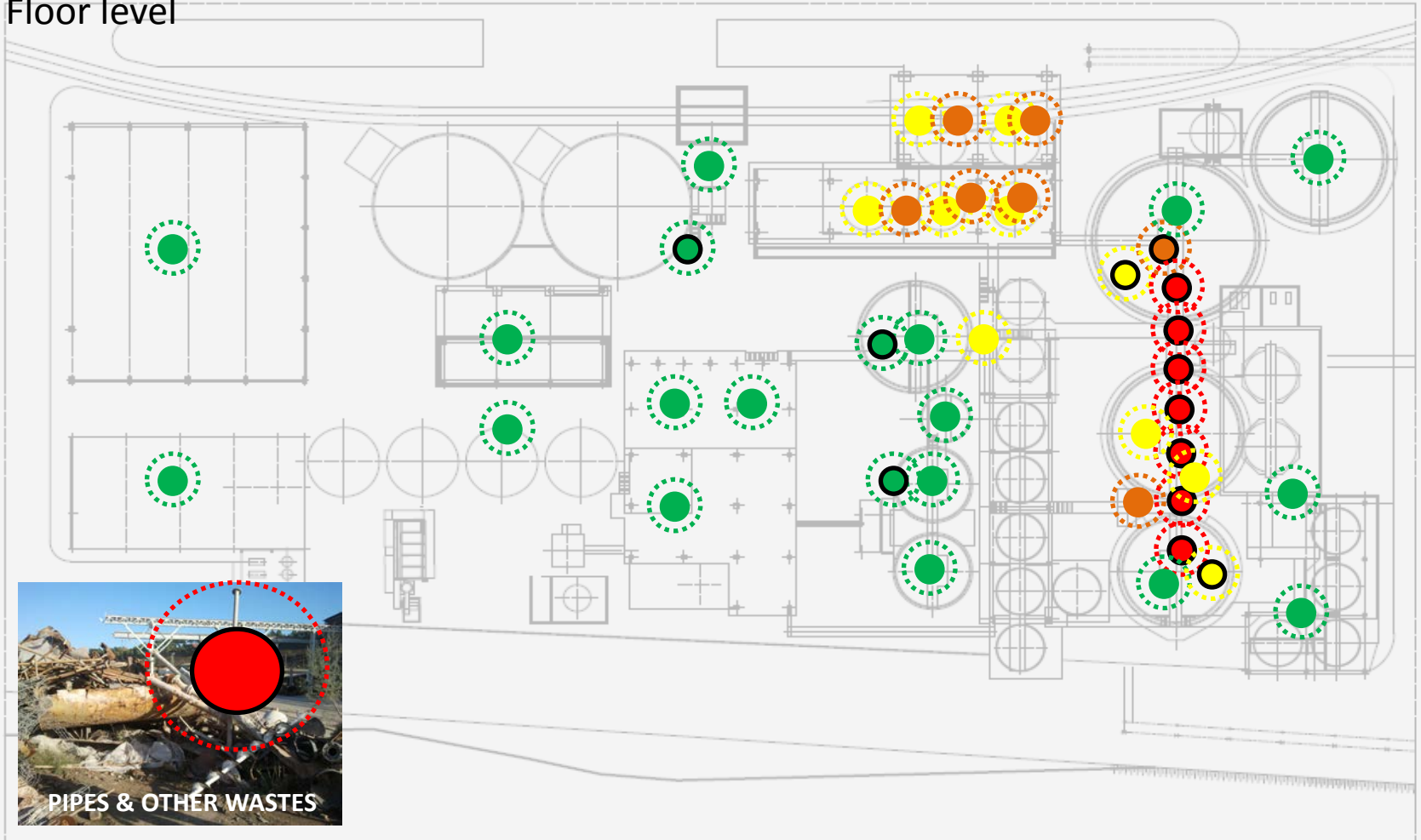
- Industrial production.
- Radionuclide concentration in products and sub-products.
- General fluxes of radionuclides.

Necessary to:

- Know the radionuclide behaviour within the industrial process.
- Estimate the amount of radioactivity in inputs and outputs.
- See the potential accumulation of radionuclides in the plant.

Gamma dose rates: AREA DOSIMETRY

○ Floor level



Gamma dose rates: HIGH EXPOSURE AREAS

Identification of two areas with HIGH GAMMA EXPOSURE:

- Reactors



- Pipes



CLEANING AND
MAINTENANCE?

Gamma dose rates: CLEANING & MAINTENANCE



Gamma dose rates: CLEANING & MAINTENANCE

1st PERSONAL DOSIMETER: < 0,1 mSv

2nd PERSONAL DOSIMETER: < 0,1 mSv

3rd PERSONAL DOSIMETER: < 0,1 mSv

4th PERSONAL DOSIMETER: < 0,1 mSv

5th PERSONAL DOSIMETER: < 0,1 mSv



CLEANING AND MAINTENANCE OF REACTORS/DIGESTORS DOES NOT SUPPOSE A RADIOLOGICAL RISK TO THESE WORKERS:

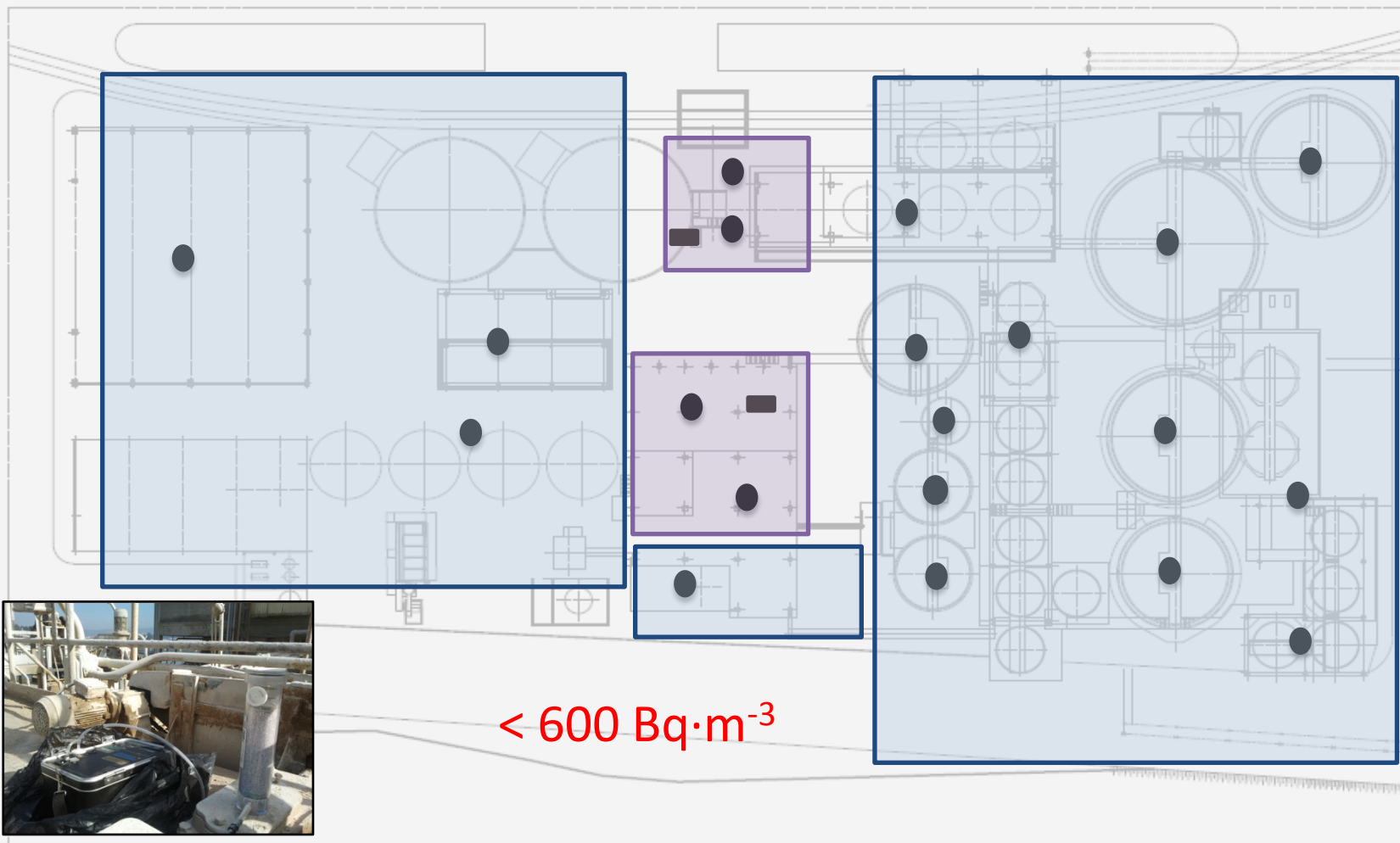
- Short time of exposure.
- High detection limit (0,1 mSv).

Inhalation dose rate: ^{222}Rn measurements

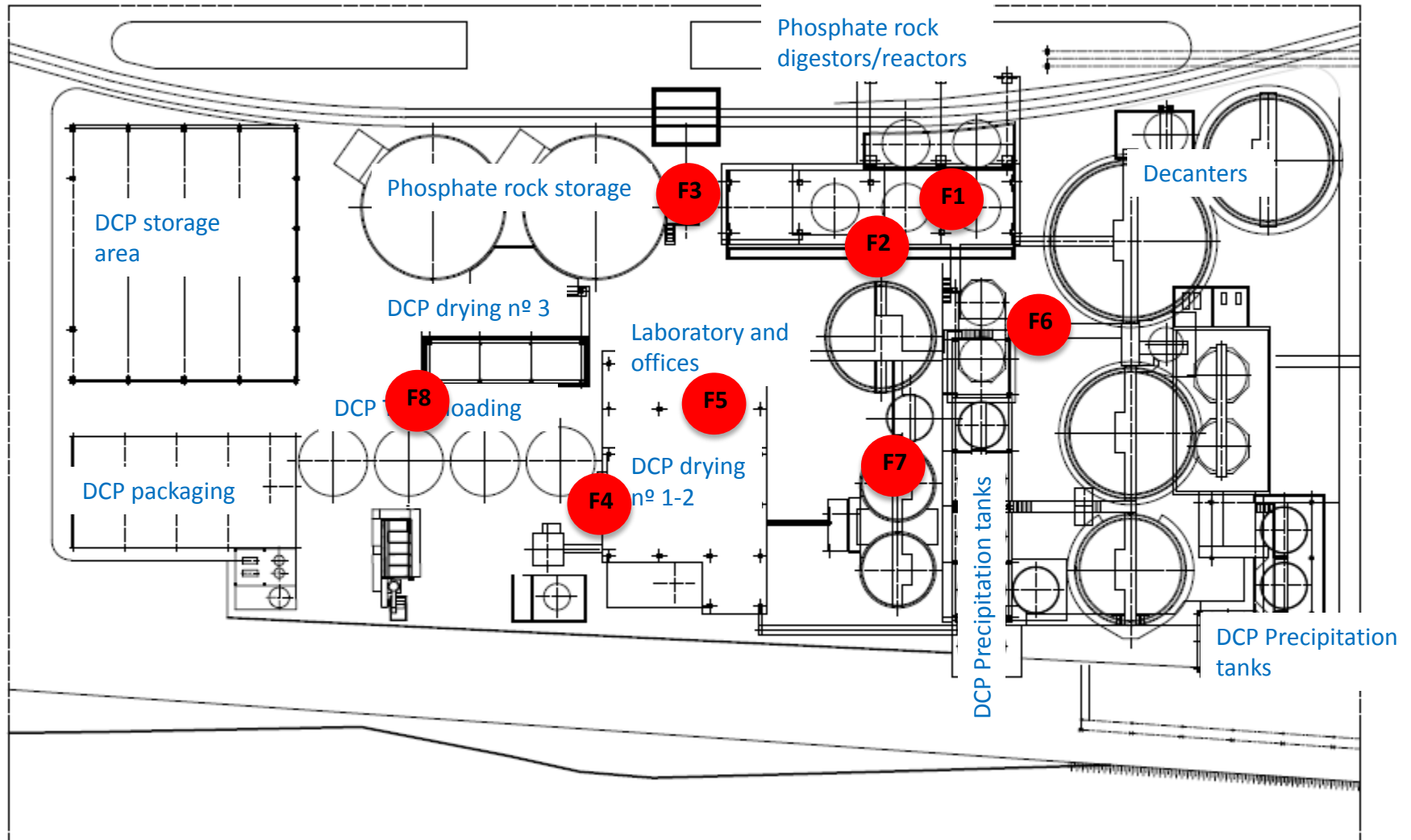
0 - 100 $\text{Bq}\cdot\text{m}^{-3}$

100 - 200 $\text{Bq}\cdot\text{m}^{-3}$

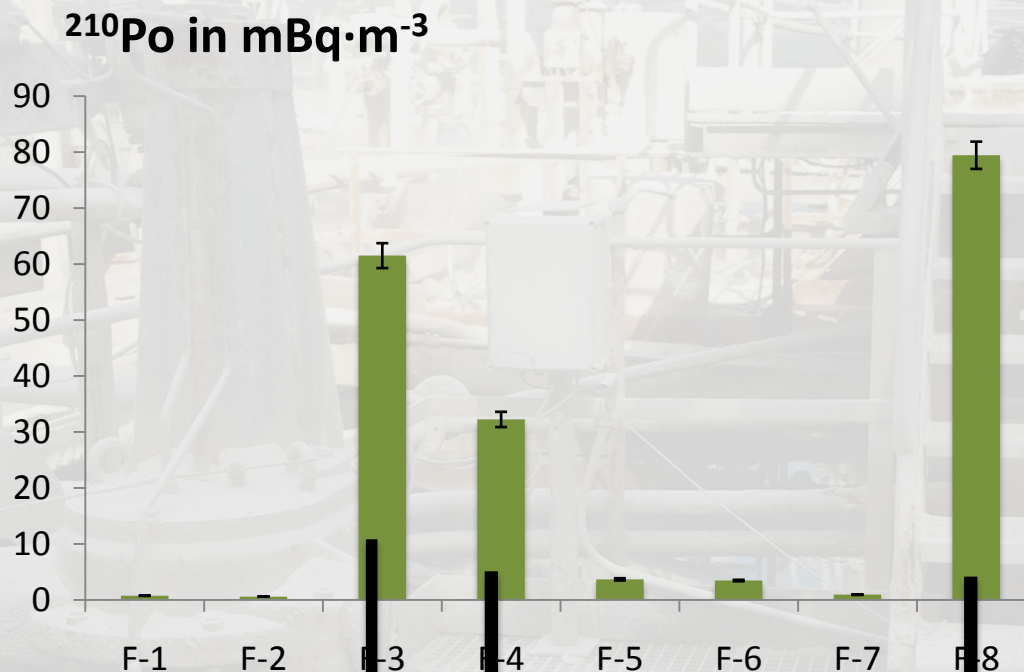
200 - 400 $\text{Bq}\cdot\text{m}^{-3}$



Inhalation dose rate: DUST CHARACTERIZATION



Inhalation dose rate: DUST CHARACTERIZATION



Phosphate rock arrival and storage to the plant

DCP package and truck loading

^{210}Po Inhalation dose rate (preliminary results)

$$E (mSv \cdot y^{-1}) = C_i \cdot V \cdot t \cdot DCC_{i(inh)}$$

C_i = Concentration of ^{210}Po in air ($\text{Bq}\cdot\text{m}^{-3}$)

V = Breathing rate ($1,2 \text{ m}^3\cdot\text{h}^{-1}$)

t = Residence time of employees at the workplace (2000/year)

$DCC_{i(inh)}$ = Dose conversion factor for ^{210}Po (if inhaled) ($\text{Sv}\cdot\text{Bq}^{-1}$)

Filter number	Description	mSv·y ⁻¹		
F-1	Close to digestors	0,0040	±	0,0002
F-2	Digestors cleaning (floor 0)	0,0032	±	0,0002
F-3	PR arrival and storage	0,32	±	0,01
F-4	DCP packaging	0,170	±	0,007
F-5	Offices	0,019	±	0,001
F-6	On production line	0,018	±	0,001
F-7	On sludges line	0,0050	±	0,0003
F-8	DCP truck loading	0,42	±	0,01

DOSE ASSESSMENT TO WORKERS: things to take into account....

Residence time at each area within the plant.

Temporal variability in:

- Industrial production.
- Radionuclide concentration in products and sub-products.
- General fluxes of radionuclides.

Security measures: reduce the dose considerably.

Formation to workers.

ANUAL DOSE: $< 1 \text{ mSv}\cdot\text{y}^{-1}$

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CONCLUSIONS

RADIOLOGICAL CHARACTERIZATION:

- Products and by-products: $< 1000 \text{ Bq}\cdot\text{kg}^{-1}$ ^{226}Ra , ^{210}Pb , ^{210}Po .
- Temporal **variability** of radionuclide concentration (depending upon production volume).
- Variability within fluxes of radionuclides but **good Output to Input Ratio** when averaging the 8 sampling weeks.
- About **40% of ^{226}Ra** is enhanced within the production process.

DOSE ASSESSMENT TO WORKERS:

- In general $< 0,5 \text{ uSv}\cdot\text{h}^{-1}$ except specific areas (reactors and pipes: up to **50 $\text{uSv}\cdot\text{h}^{-1}$**).
- Clearence and maintenance of reactors/digestors **does not** suppose a radiological risk due to low time of exposure.
- Low ^{222}Rn concentrations ($< 600 \text{ Bq}\cdot\text{m}^{-3}$).
- Potential high dose **of inhalation** (^{210}Po) in two specific areas.
- With proper formation to workers and simple security measures $< 1\text{mSv}\cdot\text{y}^{-1}$.

FUTURE WORK

FINAL DOSE ASSESSMENT, considering:

- ^{226}Ra , ^{210}Pb and ^{210}Po in dust.
- Residence time of each employee at each area.
- Cleaning and maintenance of decanters and tubing.

WASTES MANAGEMENT:

1. Pipes and tubing

2. Press filters



Thank you!

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